UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION I
475 ALLENDALE ROAD
KING OF PRUSSIA, PA 19406

January 28, 2002

Mr. K. Heider
Vice President - Operations and Decommissioning
Connecticut Yankee Atomic Power Company
362 Injun Hollow Road
East Hampton, CT 06424-3099

SUBJECT: NRC INTEGRATED INSPECTION REPORT 50-213/01-003

Dear Mr. Heider:

On January 11, 2002, the NRC completed an inspection at the Haddam Neck Plant which began on September 22, 2001. The findings of the inspection were discussed with Mr. Noah Fetherston, and others by telephone on January 17, 2002. The enclosed report presents the results of that inspection.

Your quality assurance program, security program, and radiation protection and radioactive waste management activities were inspected during this sixteen-week inspection period. The inspection consisted of selective examinations of procedures and representative records, interviews with personnel, and observations by the inspectors. Your programs were considered to be appropriately implemented and no safety concerns were identified. Effective programs for protecting the safety of workers during residual heat removal system decommissioning activities and shipping of the reactor head and emergency diesel generators were noted.

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be placed in the NRC Public Document Room (PDR) and will be accessible from the NRC Web site at http://www.nrc.gov/NRC/ADAMS/index.html. No reply to this letter is required.

Sincerely,

/RA/

Ronald R. Bellamy, Chief
Decommissioning and Laboratory Branch
Division of Nuclear Material Safety

Docket No. 50-213
License No. DPR-61

Enclosure:
NRC Inspection Report No. 50-213/01-003
Connecticut Yankee Management Meeting Handout

cc w/encl:

R. Mellor, President and Chief Executive Officer
T. Bennet, Vice President and Chief Financial Officer
N. Fetherston, Site Manager
R. M. Mitchell, Unit Manager
K. Smith, Communications Manager
G. van Noordennen, Regulatory Affairs Manager
R. K. Gad, III, CYAPCO Counsel
R. Bassilakis, Citizens Awareness Network
J. Brooks, CT Attorney General Office
T. Bondi, Town of Haddam
E. Woollacott, NEAC
State of Connecticut SLO

U.S. NUCLEAR REGULATORY COMMISSION
REGION I

Docket No.:	50-213
License No.:	DPR-61
Report No.:	50-213/01-003
Licensee:	Connecticut Yankee Atomic Power Company (CYAPCO) P. O. Box 270 Hartford, CT 06141-0270
Facility:	Haddam Neck Station
Location:	Haddam, Connecticut
Dates:	September 22, 2001 through January 11, 2002
Inspectors:	Jason Jang, Sr. Health Physicist Laurie Peluso, Health Physicist John Wray, Health Physicist
Approved by:	Ronald Bellamy, Chief, Decommissioning and Laboratory Branch (D&LB) Division of Nuclear Materials Safety (DNMS)

EXECUTIVE SUMMARY

Haddam Neck Station
NRC Inspection Report No. 50-213/01-003

This routine inspection included aspects of licensee activities regarding dismantlement and decommissioning of the facility. The report covers a sixteen-week period of inspection by regional NRC personnel, and includes reviews and assessments of the quality assurance organization, security enhancement activities, cold weather operations preparations, projected offsite dose calculations from liquid releases, and plant support activities related to radiation protection and the radioactive waste shipping programs.

Decommissioning Operations

The licensee established an adequate program to maintain the operability of systems and equipment important to safety during the cold weather season.

The licensee maintained effective quality assurance and corrective action programs and performed very good audits and assessments to help self-identify and correct issues and problems.

Timely and effective security enhancements required by NRC Safeguards Advisories and affirmed in a Confirmatory Action Letter dated October 25, 2001, were adequately implemented.

Decommissioning Status of Facilities and Equipment

Dismantlement and removal of equipment and components continued to be conducted in a safe and efficient manner in accordance with work packages and release survey plans.

Plant Support and Radiological Controls

The licensee implemented an effective radiation protection program for radiologically significant work activities. Personnel radiological exposures appear to be trending down.

The licensee maintained an adequate program for releasing radioactive liquid waste to the environment. The licensee maintained an adequate program for liquid release monitor calibrations, release discharge permits, and offsite dose calculations .

The licensee maintained an adequate program for radioactive waste shipping. Shipments of large components continue to be appropriately controlled and documented.

The licensee has established adequate soil sampling procedures for the Final Status Survey project and has adequately implemented sample collection techniques in two survey areas. Results of samples taken in sample areas 9526 and 9528 indicated that radioactive material was present and portions of these survey areas will be reclassified.

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REPORT DETAILS

Summary of Facility Activities

The plant was maintained in a permanently shutdown condition during this inspection period. The spent fuel remains in storage in the spent fuel pool and evaluations continued for possible long-term storage of the spent fuel in dry casks onsite. Dismantlement and removal of major plant equipment and structures continued with removal of equipment from the primary auxiliary building and continuation of dismantlement activities in the turbine building. The two emergency diesel generators were dismantled and shipped offsite to a non-nuclear commercial buyer.

I. Decommissioning Operations

O1 Conduct of Operations

O1.1 Station Freeze Protection Program

a. Inspection Scope (71714)

The inspector evaluated the licensee's preparations to maintain the operability of those systems and equipment important to safety during cold weather season.

b. Observations

The inspector reviewed preventive maintenance and operations procedures, checklists and the schedule for weekly tests and checks of the equipment. The inspector observed a licensee representative conduct a required surveillance and discussed the process to resolve as-found deficient conditions. Unit heaters deployed in building areas to maintain adequate ambient temperatures as required were observed. The inspector verified that heat trace was energized where required and systems were drained where appropriate. No safety concerns were identified.

c. Conclusions

The licensee established an adequate program to maintain the operability of systems and equipment important to safety during the cold weather season.

O1.2 Quality Assurance Audits and Surveillances

a. Inspection Scope (84750)

A review was performed to evaluate the effectiveness of licensee controls in identifying, resolving, and preventing issues that degrade safety or the quality of decommissioning. The inspector evaluated the licensee's self-assessment, auditing, corrective actions, and root cause evaluations through a review of licensee documents and interviews with licensee personnel.

b. Observations and Findings

The inspector reviewed selected quality assurance audits and surveillances of licensee activities which included examination of physical security and safeguards, the quality assurance program, spent fuel pool island operations, and Independent Spent Fuel Storage Installation (ISFSI) related services. These audits and surveillances were thorough and detailed, with adequate management attention to effect timely resolution to issues raised in the audits. The inspector reviewed selected corrective action reports and determined that an appropriate threshold for initiating a condition report exists.

c. Conclusions

The licensee maintained effective quality assurance and corrective action programs and performed very good audits and assessments to help self-identify and correct issues and problems.

O1.3 Miscellaneous Security Related Activities

a. Inspection Scope (71801)

The inspector reviewed enhanced security measures and evaluated compliance to requirements and commitments.

b. Observations and Findings

On October 12, 2001, a safeguards advisory was issued to communicate certain actions necessary to enhance safety and safeguards at NRC licensed facilities. This advisory supplemented the Information Assessment Team (IAT) Advisory Update issued October 6, 2001. In addition, a Confirmatory Action Letter (CAL) was issued by Region I on October 25, 2001, to affirm discussions of safeguard enhancements agreed to by the licensee. During this inspection, the inspector verified that all applicable actions of the Advisories and the CAL were implemented as required and operational.

c. Conclusions

Timely and effective security enhancements required by NRC Safeguards Advisories and affirmed in a Confirmatory Action Letter dated October 25, 2001, were adequately implemented.

II. Decommissioning Status of Facilities and Equipment

O2 Decommissioning Status of Facilities and Equipment

O2.1 Equipment Dismantlement

a. Inspection Scope (71801)

The inspector evaluated the licensee's status of decommissioning work through discussions with cognizant licensee personnel and observations of major equipment dismantlement activities.

b. Observations and Findings

During the inspection period, the licensee continued to dismantle and remove commodities from the primary auxiliary building (PAB). The inspector observed that the dismantlement activities were completed in a safe and effective manner. Licensee's efforts during this inspection period focused on the removal of equipment and commodities from the residual heat removal (RHR) system areas of the PAB. This area was highly contaminated with extremely high levels of alpha contamination. Special controls were in place to protect workers (see section R1.1). The RHR heat exchangers were removed in a safe manner and packaged for offsite shipment.

The inspectors observed work in the turbine building (TB). The building was isolated from routine traffic, and large shears were used to dismantle piping, cable trays and other interferences. The operation required a large amount of water to cool the cutting mechanisms and the inspector discussed the provisions required to ensure that the excess water on the floor did not freeze and become a slip hazard during the winter. The licensee identified a number of near miss events during self assessments and audits of their contractor operations. The inspector stated that the findings and remedial actions to improve industrial safety during TB demolition will be reviewed following completion of the licensee's evaluation.

The licensee removed the two emergency diesel generators (D/G) and shipped them offsite to a non-nuclear buyer for reuse. The inspector observed final packaging and removal activities for the D/Gs and concluded that the dismantlement and loading operations were completed in a safe manner. (see section R1.3)

During this inspection period, the licensee removed the reactor head from containment and shipped it off site for burial at an NRC licensed radioactive material burial facility. The reactor head required special rigging for its removal from containment and an oversized truck for shipment. The inspector observed selected activities throughout the removal process. No safety concerns were identified. (See section R1.3)

c. Conclusions

Dismantlement and removal of equipment and components continued to be conducted in a safe and efficient manner in accordance with work packages and release survey plans.

III. Plant Support and Radiological Controls

R1 Radiological Protection Controls

R1.1 Occupational Exposure Controls

a. Inspection Scope (83750)

The inspector reviewed the licensee's routine radiation protection surveillances and planning for significant decommissioning activities to determine adequacy of the licensee's radiation protection program under various conditions. The inspector also interviewed selected radiation protection supervisors and staff.

b. Observations and Findings

The inspector observed planning, preparation, and conduct of work activities in radiologically challenging environments. The RHR area was highly contaminated and surveys indicated a alpha to beta/gamma ratio of 1:1. This environment required special radiological controls to ensure workers were not exposed to concentrations of radioactive material greater than 10CFR20 limits and that no uptakes of alpha contamination occurred. The inspector observed work activities in the PAB and noted good use of teledosimetry. Discussions with workers and radiation protection technicians indicated that individuals were cognizant of the Radiation Work Permit (RWP) requirements and the radiological environment in which they were working. Lead technicians maintained the appropriate level of control over activities for which they were responsible. The inspector verified that the requirements of the RWP and As Low As Reasonably Achievable (ALARA) review, including respiratory protection and engineering controls, were satisfied. The inspector reviewed random whole body counts of workers performing decommissioning activities in the RHR area and noted that no internal uptakes of radioactive material was identified. No safety concerns were identified.

The inspector observed work ongoing in the reactor cavity involving cutting and removal of the reactor segmentation equipment. The inspector noted good use of underwater cutting apparatus and appropriate radiological controls of work activities on the bridge. The reactor segmentation equipment remained highly contaminated with potential for Greater Than Class C (GTCC) material becoming a personnel hazard. The inspector reviewed with cognizant licensee representatives the use of A-43 liners in the reactor segmentation project. These liners were obtained to collect used garnet from the vessel cutting operation. The liners were designed to be dewatered underwater and had a Certificate of Compliance (CofC) to be buried at the Barnwell radioactive material burial facility. The inspector verified that, although a CofC existed to permit burial at Barnwell, the licensee will not dispose of them in this manner. They were segmented and shipped for burial in appropriately certified containers. The licensee stated that the A-43s never functioned as designed in that they were never capable of being dewatered underwater. However, shortly after the reactor segmentation project began, the A43 liners were used as underwater storage containers for the large volume of garnet generated by the segmentation operations. The inspector verified by review of documentation and interviews with cognizant licensee personnel that the A43 liners were used in a safe manner and were not used in an inappropriate or non-licensed manner. No safety concerns were identified.
The inspector reviewed the maintenance and calibration records for selected survey and monitoring instruments that are located in the turbine building. The personnel contamination monitor (PCM-1B); small article monitor (SAM 9); Bicron NE; Electra; Ludlum 2200; Eberline E-140;

Eberline RM-14. Calibrations and performance checks were performed according to the procedures specific for each type of survey instrument. The results were within the licensee's acceptance criteria.

The inspector attended ALARA committee meeting number 2001-22. The licensee discussed a new reactor cavity filtration system and controls for other radiologically significant work. The inspector noted that much of the discussion was led by non-health physics personnel, indicating good ownership of radiation protection activities by the craft. The licensee indicated that, although dose goals are approaching 80% of the goals, the trend is downward, and they believe that the ownership of ALARA by the craft was a significant reason for the improvement. The inspector acknowledged the observed downward trend in site doses for the past three months. The inspector stated that review of the final site personnel exposures for the year 2001 will be reviewed during a future inspection. No safety concerns were identified.

c. Conclusions

The licensee implemented an effective radiation protection program for radiologically significant work activities. Personnel radiological exposures appear to be trending down.

R1.2 Radioactive Liquid Effluent Control Program

a. Inspection Scope (84750)

The inspector evaluated the effectiveness of the licensee's liquid effluent discharge program by reviewing the release of contaminated water from the "A" Recycle Test Tank. This review consisted of an examination of discharge papers, release procedures, monitor calibration and setpoint determinations, and offsite dose calculations completed to assure compliance with the Radiological Effluent Monitoring and Offsite Dose Calculation Manual (REMODCM).

b. Observations and Findings

The inspector reviewed discharge paperwork for liquid release L-24 on November 29, 2001, and Operations Procedure entitled "Discharge of Liquid Radioactive Waste" . The Chemistry Department liquid waste release procedure was also reviewed and requirements controlling tank recirculation times and sample analyses were verified. The inspector observed that the liquid waste discharge monitor (R22) had been relocated away from decommissioning work being conducted in the PAB. The inspector verified that the monitor was recalibrated following relocation. Calibration results were within the licensee's acceptance criteria provided in the procedures, and were evaluated and tracked by the chemistry staff. The inspector verified the release setpoint was appropriate and was not exceeded at anytime during the release.

After the tank contents were released, the licensee identified a deficiency in their release procedures. The licensee identified that the radioactive liquid effluent dose model provided in the REMODCM did not specify the duration of the dilution water pump operation at the termination of a radioactive liquid release. This could impact the projected public dose calculation methodology because the dilution factor might change depending on when the dilution water pump was turned off.

The inspector conducted an independent verification, using the NRC PCDOSE code, of the licensee's capability for calculating projected public dose, listed in the REMODCM. The NRC

PCDOSE code is based on Regulatory Guide (RG) 1.109, Calculation of Annual Doses to Man from Routine Release of Reactor Effluents for the Purpose of Evaluating Compliance with 10 CFR Part 50, Appendix I. The inspector evaluated the licensee's computer code by using site specific parameters and monthly release information.

The results of the radioactive liquid release pathway intercomparison were acceptable. The adult total body doses for November 2001 by the NRC and by the licensee were 4.31E-2 mrem and 4.82E-2 mrem, respectively. (**Acceptance Criteria**: **within a factor of 2,** NRC Baseline Inspection Procedure 72211.01, Radioactive Gaseous and Liquid Effluent Treatment and Monitoring Systems.)

The licensee's projected public dose calculations were based on fish and invertebrate pathways. (There was no drinking water pathway.) The adult annual consumption rate for fish and invertebrates were 21 kg and 5 kg, respectively, as recommended by RG 1.109. The inspector stated that the licensee can use the actual annual fish and invertebrate consumption rates, as site specific values. For example, if the public did not consume fish and invertebrates caught in the discharge canal, then the projected public doses would be zero. The licensee stated that they will evaluate the actual annual fish and invertebrate consumption rates and will use these values in the projected public dose calculations. No safety concerns were identified. The inspector had no further questions.

c. Conclusions

The licensee maintained an adequate program for releasing radioactive liquid waste to the environment. The licensee maintained an adequate program for liquid release monitor calibrations, release discharge permits, and offsite dose calculations .

R1.3 Radioactive Waste Management and Transportation

a. Inspection Scope (86750)

The inspector reviewed selected radwaste shipments and determined compliance to NRC and Department of Transportation (DOT) regulations. This inspection consisted of a review of records and observation of packaging and surveillance activities associated with a shipment.

b. Observations and Findings

The inspector observed preparations and loading activities associated with radwaste shipment 2001-261, which was a shipment of filters from the reactor segmentation project in a steel liner inside a 8-120 type B cask. Following removal of the liner from containment, it was placed into the cask shackled onto the bed of a qualified truck. The inspector observed the loading of the liner and verified that the pressure test of the cask seal met certificate requirements. No safety concerns were identified. The inspector reviewed the shipping papers associated with this shipment and interviewed licensee radwaste transportation personnel and the driver of the vehicle. All required surveys, notifications, and instructions were present. No discrepancies with regulatory requirements were identified. The inspector conducted an independent survey of the truck prior to leaving the site and verified dose rates did not exceed applicable DOT limits.

During this inspection period, the licensee also shipped the reactor head. This shipment was a Class A low specific activity (LSA) II shipment and contained 9690 millicuries of total radioactivity.

The inspector reviewed the waste manifest, bill of lading, driver and special instructions for exclusive use controls, the package characterization report, radiation surveys, and the Special Nuclear Material (SNM) Exemption Certification. No discrepancies were identified. The inspector reviewed the licensee's Technical Support Document entitled, "Characterization of Reactor Head and Associated Components", which determined the isotopic mix for this shipment. No safety concerns were identified. The route of the shipment was reviewed and no concerns were identified. The reactor head was received and buried at Envirocare in Utah on schedule and without any incidents .

During this inspection period, the emergency diesel generators (D/G) were packaged and shipped to an offsite buyer. The inspector observed the packaging of the D/Gs and reviewed the radiological survey plan for their release for unrestricted use. The survey plan required 100% survey of disassembled items removed from the generators, survey of all flanges and valves, and 100% of the surfaces of the D/Gs. Any system liquid (i.e. oil, antifreeze) was required to be analyzed by gamma spectroscopy. The plan was comprehensive and complete. The inspector reviewed survey results and determined that no radioactive material was identified (the D/Gs were located outside the radiologically controlled area of the plant and should not have come in contact with radioactive material). No safety concerns were identified.

c. Conclusions

The licensee maintained an adequate program for radioactive waste shipping. Shipments of large components continue to be appropriately controlled and documented.

R1.4 Final Status Survey (FSS) Program

a. Inspection Scope (83801)

The inspector reviewed licensee's decommissioning procedures and associated documentation for the Final Status Survey (FSS) program to determine compliance with regulatory requirements and site commitments. Soil sample collection techniques were observed and split samples were obtained.

b. Observation and Findings

The inspector verified the calibration records and performance checks for the survey instruments (E-600/SPA-3) used for scanning survey area 9526 (scan areas 4&5). The inspector reviewed calibration procedures and noted that the calibrations and performance checks were performed appropriately and the results were within the acceptance criteria.

The inspector reviewed additional procedures for performing specific functions of the FSS program. Sample collection techniques were adequate to minimize cross contamination. Sampling procedures were appropriately followed.

Split samples were obtained and shared with the licensee, NRC and the Connecticut DEP. Results of the licensee's soil samples indicated the presence of low levels of Co-60 and/or Cs-137. Sample analysis results by the Connecticut Department of Environmental Protection (DEP) confirmed the presence of radionuclides in these sample locations. The licensee had designated the survey areas from which these positive samples had been taken as Class 3 areas and did not expect positive indication of radioactive material. The licensee stated that they will reclassified portions of

survey areas 9526 and 9528 and increase sampling as appropriate. The inspector concurred with the licensee's action at this time and will closely monitor results of future sampling analyses.

c. Conclusions

The licensee has established adequate soil sampling procedures for the Final Status Survey project and has adequately implemented sample collection techniques in two survey areas. Results of samples taken in sample areas 9526 and 9528 indicated that radioactive material was present and portions of these survey areas will be reclassified.

R8 Status of Previous Radiation Protection Items

a. Inspection Scope (71801)

The inspector reviewed documentation packages that had been prepared by the licensee to support closure of a radiation protection issue.

b. Observations and Findings

Closed (URI 2001-002-01): On April 5, 2001, the licensee issued condition report (CR) 01-0123 to investigate an unexplained correlation between use of components in the liquid waste release systems and the external containment mat sump (ECS) tritium activity. The licensee also investigated a possible system leak into the sub-surface environment that could lead to an uncontrolled, unmonitored release of radioactivity (tritium) to the environment. The investigation included an independent review of effluent data, ECS tritium activity, and a video inspection of the inside of the service water discharge header. The licensee's investigation addressed site characterization and liquid effluent pathways, and provided a completed document which supports effectively controlled and monitored effluent releases of tritium.

c. Conclusions

The inspector reviewed the licensee's documentation including a video tape and determined that the investigation was complete and supports closure of a potential uncontrolled, unmonitored release of tritium to the environment. This item is closed.

V. Management Meetings

X1 Exit Meeting Summary

The inspectors presented the inspection results to members of licensee management periodically during the inspection, and during a teleconference with the site manager and others on January 17, 2002. The licensee acknowledged the findings presented by the inspectors. The inspector reviewed with the licensee whether any materials examined during the inspection should be considered proprietary. No proprietary information was identified.

X2 Other Meetings

Senior licensee representatives met with Region I staff and management and staff from NRR on January 10, 2002, in the Region I office. The meeting was open to the public. The Haddam Neck site, as well as the state offices of Massachusetts and Connecticut were linked to the meeting by telephone. Recent decommissioning activities were discussed for Haddam Neck including site safety performance, fuel storage and transfer preparations and activities, security enhancements, and status of the License Termination Plan (LTP). A copy of the licensee's handout is enclosed.

PARTIAL LIST OF PERSONS CONTACTED

J. Allen, Project Manager, Duratek
* J. Bourassa, Safety Oversight Manager
M. Cavanaugh, Communications Manager
E. Darois, Health Physicist, Bechtel
J. DeLawrence, Technical Support Specialist
D. Drulard, Site Construction Manager, Bechtel
H. Farr, Radiological Engineer, Bechtel
* N. Fetherston, Site Manager
* M. Firsick, Connecticut DEP
K. Gavin, Project Field Engineer, Bechtel
R. E. Gault, Rad Protection Specialist
K. Heider, Vice President Operations and Decommissioning
* K. Jackson, Assistant Project Manager, Bechtel
* J. Lynch, Construction Oversight Manager
* R. McGrath, HP and Waste Oversight Manager
W. McConnell, Assistant Waste Manager, Bechtel
D. Montt, Chemistry Oversight
* R. Mitchell, Unit Manager
* F. Perdomo, Regulatory Affairs
* M. Powers, Construction Oversight
* R. Prunty, Licensing, Bechtel
G. Seckenger, Offsite Material Recovery Program
* E. Sergent, Nuclear Safety
E. Shyloski, Project Manager, Bechtel
* J. Tarzia, Radiation Protection and Chemistry Manager, Bechtel
R. Vallem, Waste Management Supervisor, Bechtel
G. van Noordennen, Regulatory Affairs Manager
* S. Webster, Licensing, Bechtel
A. Yates, Chemistry Supervisor

*Denotes attendance at the telephonic exit meeting held on January 17, 2002.

INSPECTION PROCEDURES USED

IP 71714:	Cold Weather Preps
IP 71801:	Decommissioning Performance and Status Review
IP 83801:	Inspection of Final Surveys at Permanently Shutdown Reactors
IP 83750:	Occupation Radiation Exposure Controls
IP 84750:	Radwaste Treatment/Effluent and Environmental Monitoring
IP 86750:	Solid Radwaste Management and Transportation

ITEMS OPEN, CLOSED, AND DISCUSSED

Open

NONE

Closed

2001-002-01	URI	Potential Uncontrolled, Unmonitored Release to Groundwater Wells

Discussed

NONE

LIST OF ACRONYMS USED

ALARA	As Low As Reasonably Achievable
CofC	Certificate of Compliance
CAL	Confirmatory Action Letter
CFR	Code of Federal Regulations
CR	Condition Report
CY	Connecticut Yankee
CYAPCO	Connecticut Yankee Atomic Power Company
D&LB	Decommissioning and Laboratory Branch
DEP	Department of Environmental Protection
DNMS	Division of Nuclear Materials and Safety
ECS	External Containment Mat Sump
DOT	Department of Transportation
D/G	Emergency Diesel Generator
FSS	Final Status Survey
GTCC	Greater than Class C
HP	Health Physics
IAT	Information Assessment Team
ISFSI	Independent Spent Fuel Storage Installation
LSA	Low Specific Activity
LTP	License Termination Plan
NCV	Non-Cited Violation
NRC	Nuclear Regulatory Commission
PAB	Primary Auxiliary Building
QA	Quality Assurance
RCA	Radiologically Controlled Area
REMODCM	Radiological Effluent Monitoring and Offsite Dose Calculation Manual
RWPs	Radiation Work Permits
RG	Regulatory Guide
RHR	Residual Heat Removal System
SNM	Special Nuclear Material
TB	Turbine Building
URI	Unresolved Item